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**ATOMEXPO 2018
Round Table on
International Scientific
cooperation and
advanced research
infrastructures**

**JHR Project: a new modern 21st
century Material Testing Reactor
working as an international User's
Facility in support to Research
Institutes, Nuclear Industry and
Regulators**

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CEA-ICERR general representative for the IAEA
CEA –CE Cadarache (France)***

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Position of Material Testing Reactors within Nuclear R&D scheme

The Key-Role of Material Testing Reactors for Fuel and Material qualification under irradiation: R&D in support of Nuclear Power Industry

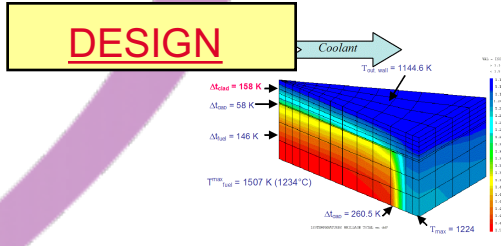
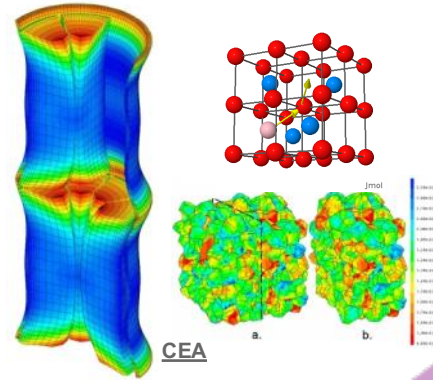


CODES Validation
QUALIFICATION Documents

BASIS RESEARCH & NUMERICAL SIMULATION

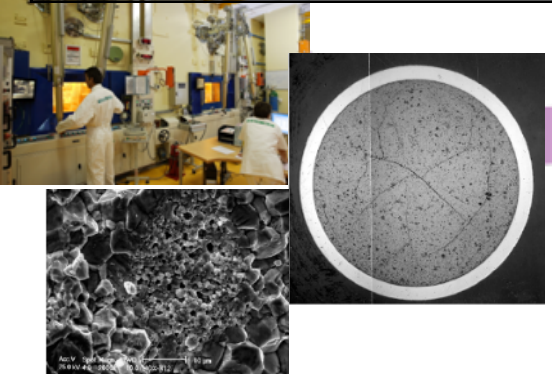


EXPERIMENTAL DATA EXPERTISE



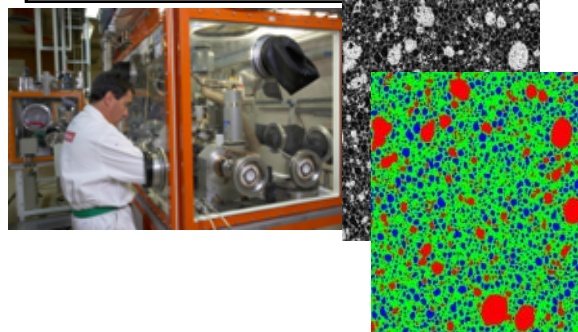
POST IRRADIATION EXAMINATIONS **SINGLE EFFECT EXPERIMENTS**

Hot laboratories for PIE



BEHAVIOUR UNDER IRRADIATION
Material Test Reactor

MANUFACTURING REFABRICATION CHARACTERIZATION
Hot laboratory



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France Strategy for Material Testing Reactor: the future JHR as an International User Facility

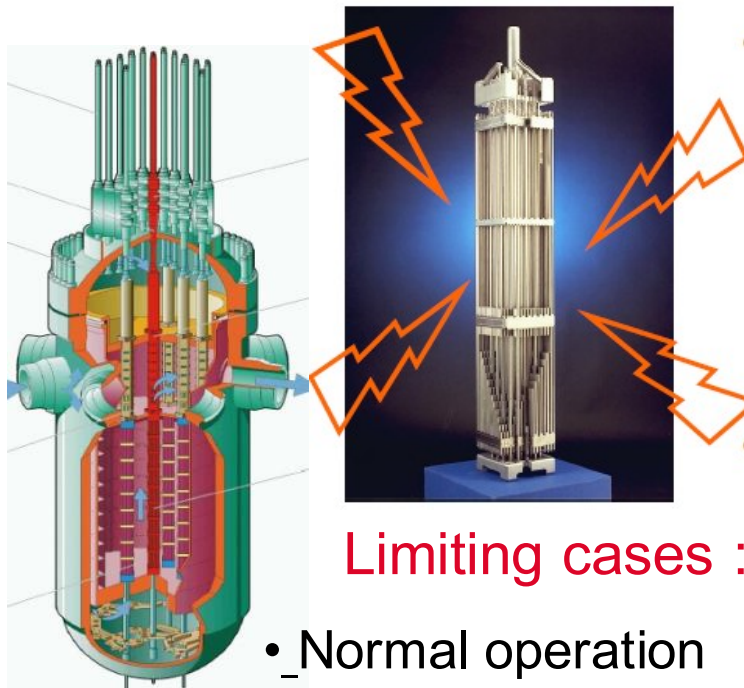
*(contribution from Dr Nicolas Waeckel
–EDF/SEPTEN)*



For EDF, 58 NPPs in France (15 in UK) means more than 10 000 F/As under irradiation at a time...



... in an aggressive environment, for more than 6 years :



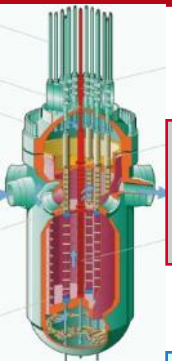
Limiting cases :

- Normal operation » FGR, Creep & Growth, clad T, ..
- Class1- 4 transients » Too much power (SCC-PCI, RIA,..)
- » Not enough cooling (LOCA, Dry-out, ..)



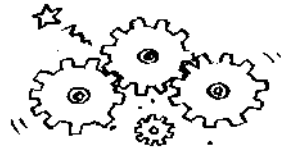
In-pile data required !

From tests results to Design Margins assessment



In-reactor Experience

Separate effect tests



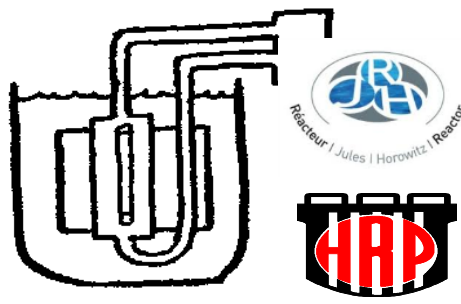
Models, Tools, Design limits, Methods

Simulation (Class 1 to Class 4 transients)

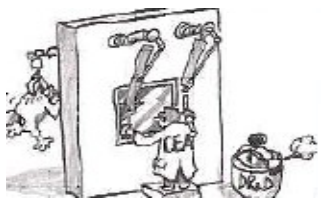


Design Margins

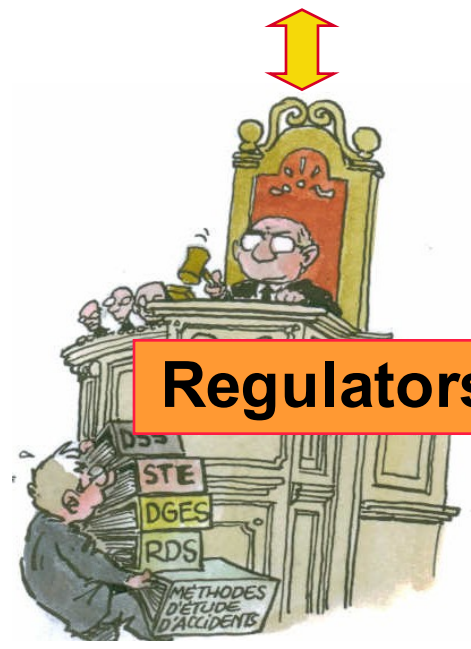
Safety Case Analysis Files



In-pile tests

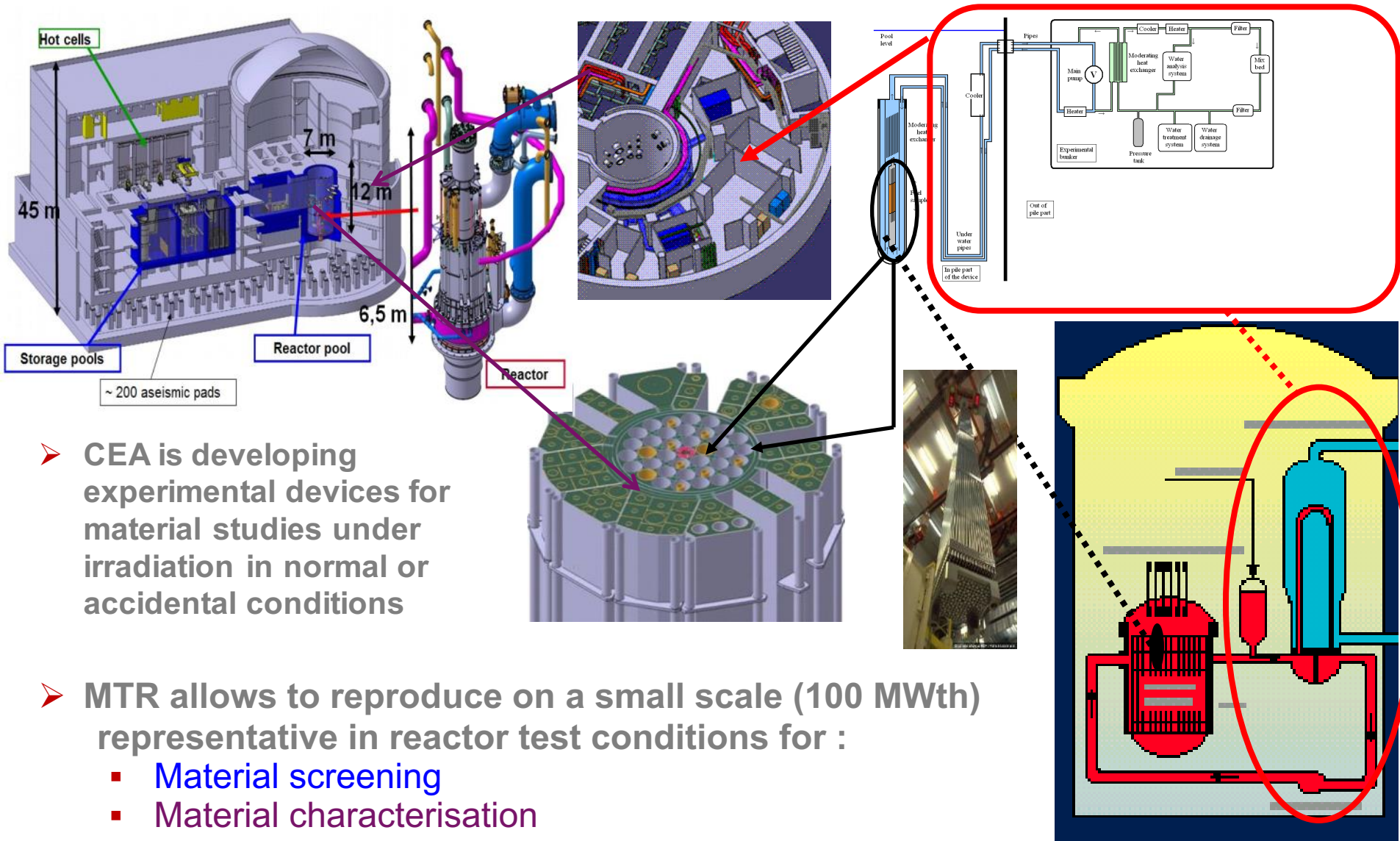


Out-of-pile tests



Regulators

The appropriate answer to the Industry needs : JHR Project



➤ CEA is developing experimental devices for material studies under irradiation in normal or accidental conditions

➤ MTR allows to reproduce on a small scale (100 MWth) representative in reactor test conditions for :

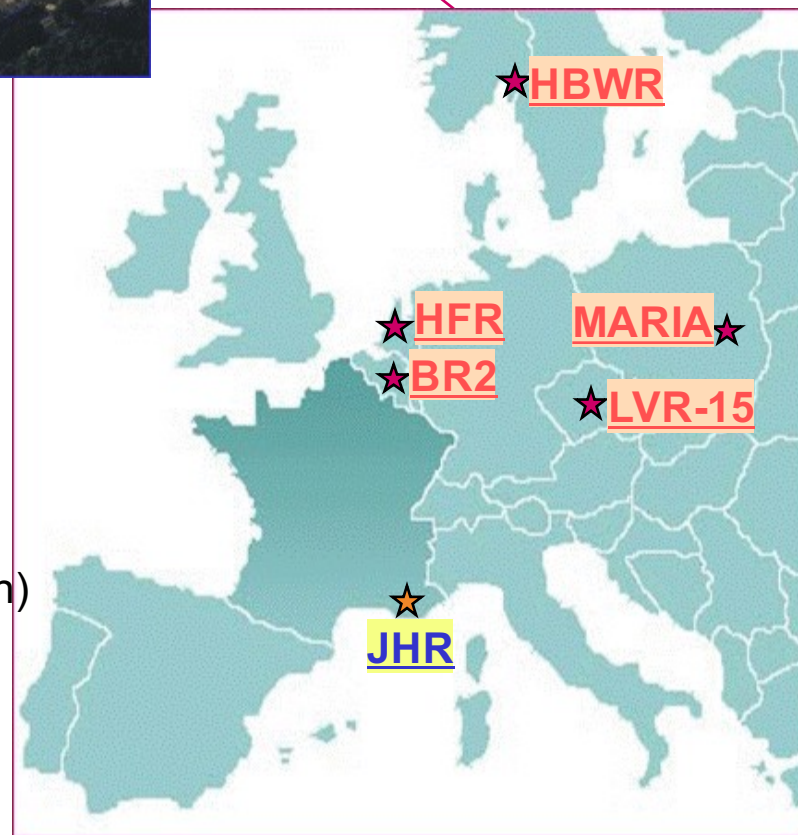
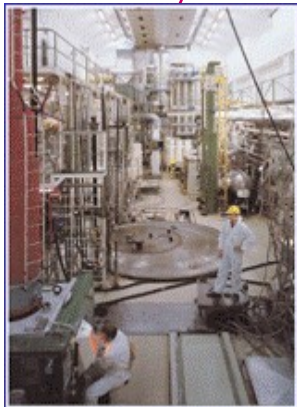
- Material screening
- Material characterisation
- Fuel element qualification

The Genesis of JHR within the European context of an Ageing fleet of MTR



Age of current E.U. main MTRs in 2018 (years)

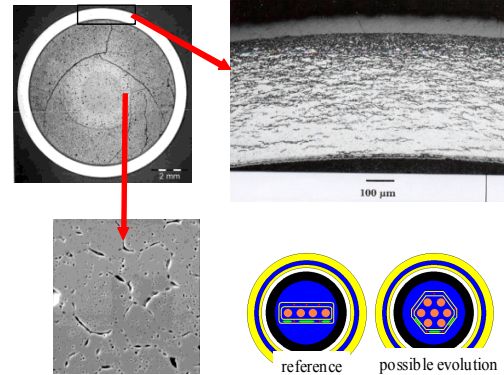
BR2 (B)	55
HALDEN (N)	60
HFR (NL)	57
LVR 15 (CZ)	60
MARIA (PO)	45
OSIRIS (F)	52 (shut down)



★ Under construction

1. R&D in support to nuclear Industry

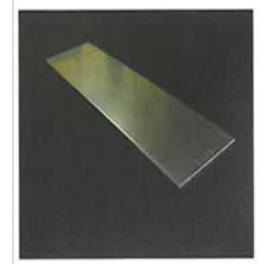
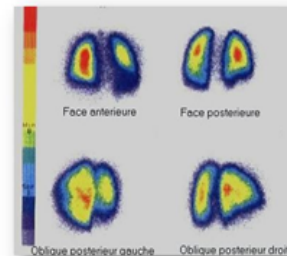
- Safety and Plant life time management (ageing & new plants)
- Fuel behavior validation in incidental and accidental situation
- Assess innovations and related safety for future NPPs



2. Radio-isotopes supply for medical application

■ ⁹⁹Mo production

- JHR will supply 25% of the European demand (today about 8 millions protocols/year)
- Up to 50% upon specific request



3. A key tool to support expertise

- Training new generations (JHR simulator, secondees program)
- Maintaining a national expertise staff and credibility for public acceptance
- **Assessing safety requirements evolution and international regulation harmonization**





JHR OPERATING RULES



JHR consortium gathers organizations which take part financially and get permanent access to JHR experimental capacities
(1 representative / organization)

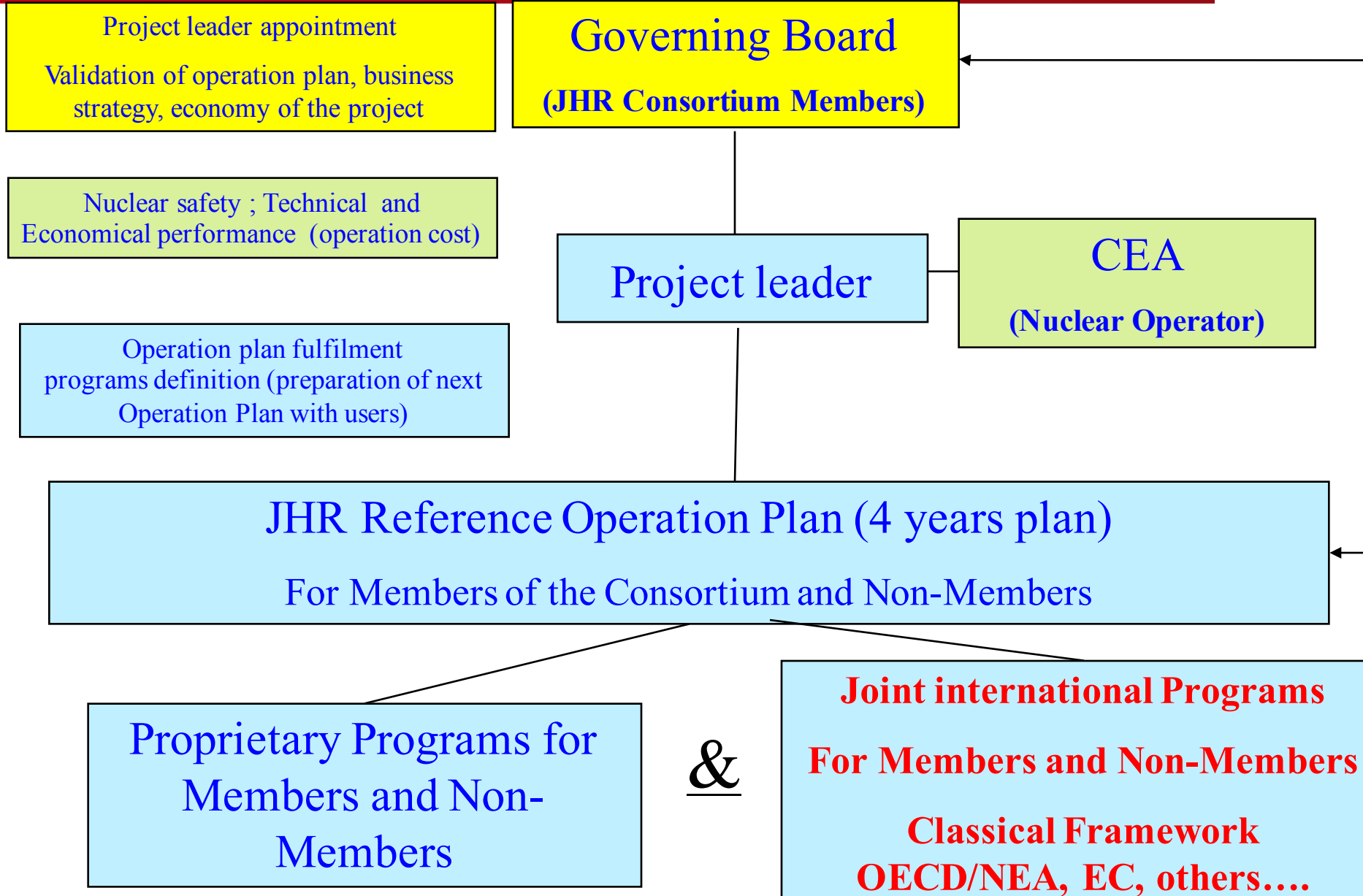
JHR Consortium current partnership: Research centers & Industrial companies



In some cases, the organization (member of the JHR consortium) is itself the representative of a national domestic consortium which gathers organizations among industry, R&D organizations, TSO, or Safety Authority



JHR GOVERNANCE



« *The activity of the JHR project will consist of:*

A) ***Proprietary Programs*** addressing services to the exclusive benefit of industrial companies or research institutes who shall have the full property of the results

B) ***The International Joint Program (IJP)*** addressing priorities common to a large community sharing the produced information within the Joint Data Basis (Data bank with all the knowledge and results carried out within the IJP implemented in the JHR capacity) »

Consortium members

JHR ORGANISATION IN OPERATING PHASE



- AREVA 10 %
- CEA 44 %
- CIEMAT 2%
- DAE 3%
- EDF 20 %
- EC-JRC 6 %
- NRI 2%
- STUDSVIK 2%
- SCK 2%
- VTT 2%
- IAEC 2%
- NNL 2%
- + ...

Votes

CONSORTIUM

Users

(Programs définition)

JHR Project Nuclear Operator

Validation of

- The Project Leader
- The budget
- The Business Strategy

CEA Operator in charge of:

- Nuclear Safety
- Technical Performance
- Economical Performance

Management of requests from non-members

- Access submitted to the Strategy Policy decided by the Governing Board and the Availability

- Cost = neutron cost + Return of Investment + margin

Cadarache Centre
(Technical Support)

CONFIDENTIALITY
Versus CEA R&D
And between Members

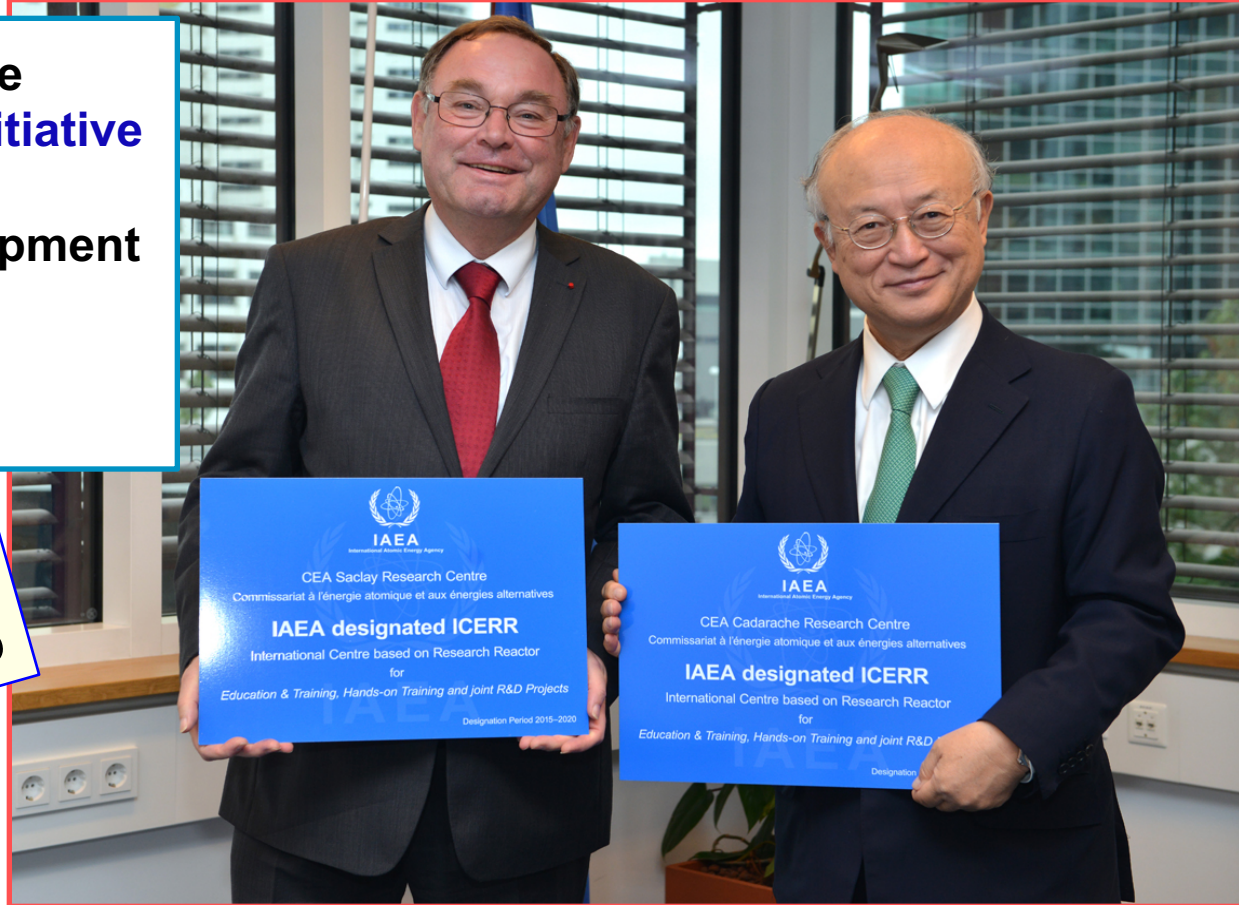
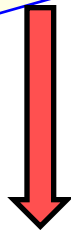
INTERNATIONAL RECOGNITION THE JHR AND ITS ANCILLARY FACILITIES AS AN IAEA- ICERR : FIRST DESIGNATION –Sept 2015

Fully compliant with the
French Capacity Building Initiative

based on 4 pillars:

- Human Resources Development
- Education & Training
- Knowledge Management
- **Knowledge Network**

IAEA ICERR labelling
obtained
on 14th September 2015



Strong CEA intention to welcome Junior and/or Senior Scientists, Nuclear Engineers, Operators, Safety Managers... within JHR teams for various topics (R&D programs, Hands-on training on equipments...)



JHR, an advanced Research Infrastructure to support nuclear industry competitive advantages:

- Better Modelling*
 - Embarked on-line instrumentation*
 - Better monitoring of irradiation conditions*
- Probably more “Costly” & “Complex” experiments*
→ More Share R&D (International Joint Programs)
(Even for “competitive market” such as ATF-type Fuel development and qualification)

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Example: Fuel program proposal for an OECD/NEA Joint Project

An initiative from the JHR Fuel Working Group



Fuel gaseous swelling during a power ramp test

Discriminate / quantify mechanisms

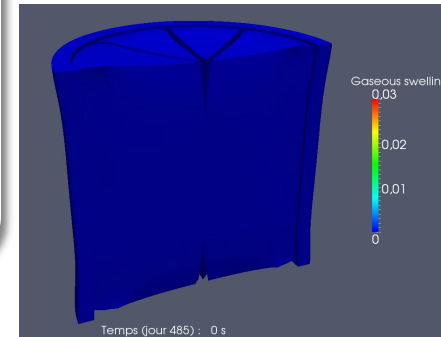
- Inducing a moderate to high load on the clad during a power transient:

↪ Fuel thermal expansion

↪ Fuel gaseous swelling

↪ Fission gas release

↪ Fuel volume change at melting



Modeling/Simulation

■ Enlarge current databases for code validation (including codes benchmarking)

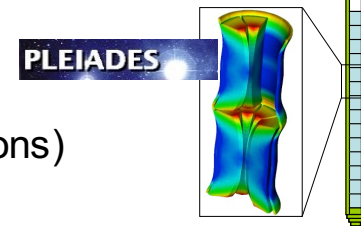
- Pre-calculation of the rod behaviour (test conditions / instrumentation specifications)

- Prediction of expected results:

↪ Fuel central T vs. LHGR, Margin to fuel central melting, Melted volume fraction at maximum LHR

↪ FG distribution, migration and release ↪ Expected clad deformation in hot/cold conditions

- Post-test comparison calculation vs. experimental results



Application to industrial needs

- Results transposition to reactor conditions for addressing issues **facilitating NPP flexible operation**

- Improve **quantification of available margins on current fuel management constraints**

- **Gain licensing data usable for new fuel products and for new licensing methodologies**

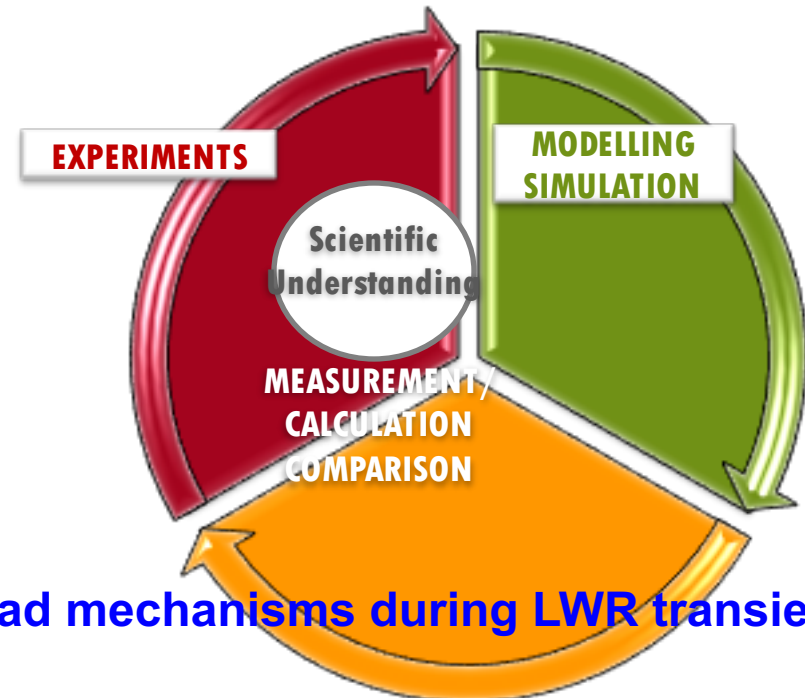
❖ Progressive introduction of advanced or new fuels and clads, such as **Accident Tolerant Fuels**

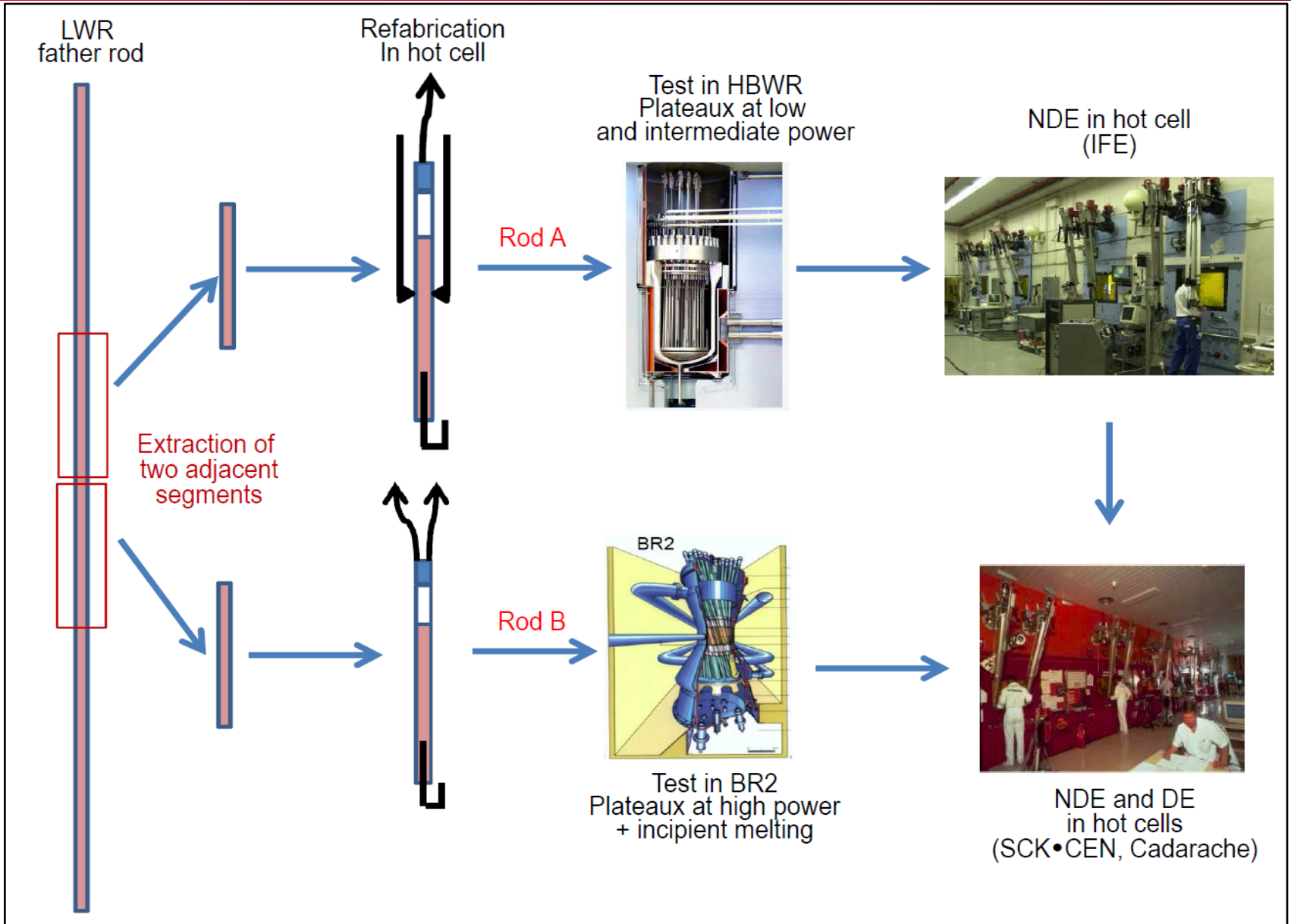
Enhanced experiments performed through international joint projects is of primary importance to successfully address these issues



Quantifying clad thermomechanical load mechanisms during LWR transients

- ❖ Focused on “long-lasting transients”, for which the slow kinetics:
 - ✓ Should not induce a risk to reach the technological failure limit of the cladding
 - ✓ May not trigger preventive safety actions of the reactor
- ❖ Discriminate clad load mechanisms thanks to a separate effect approach based on successive power plateaus with progressive increased values
- ❖ As conservative approach, incipient fuel melting at the pellet center is considered

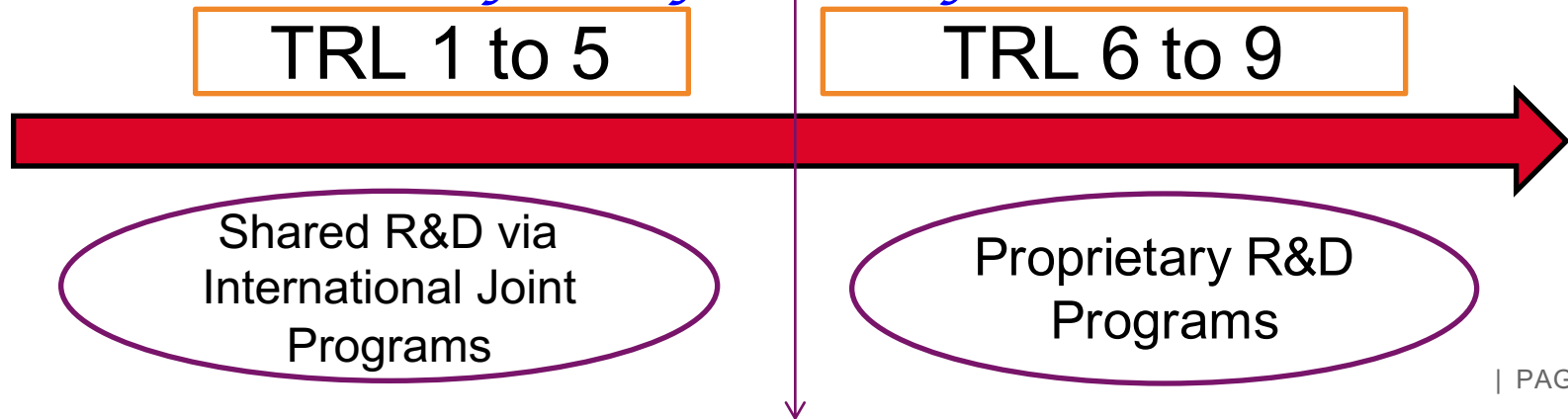


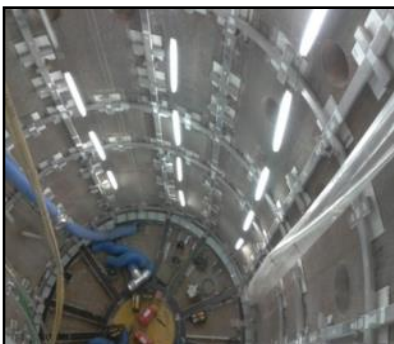


JHR, an advanced Research Infrastructure to support nuclear industry competitive advantages

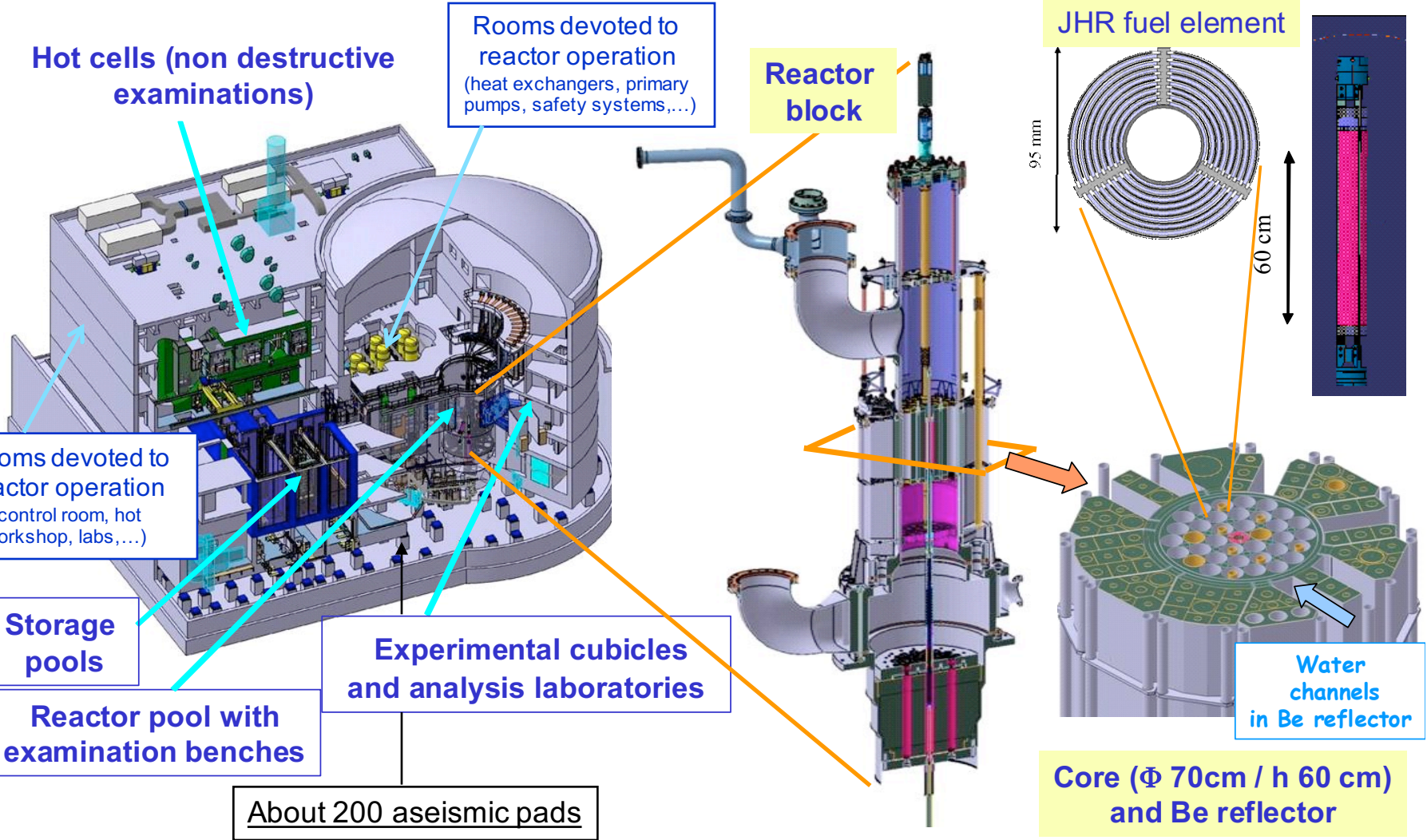
Exemple of reducing timescale for ATF-type Fuel Qualification :

*first part shared R&D (screening phase) even with several fuel vendors
Second part on proprietary programs (Fuel qualification)
with full confidentiality and IP*

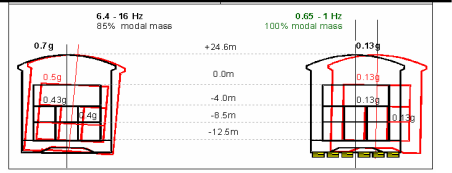




Thanks for your attention



About 200 aseismic pads

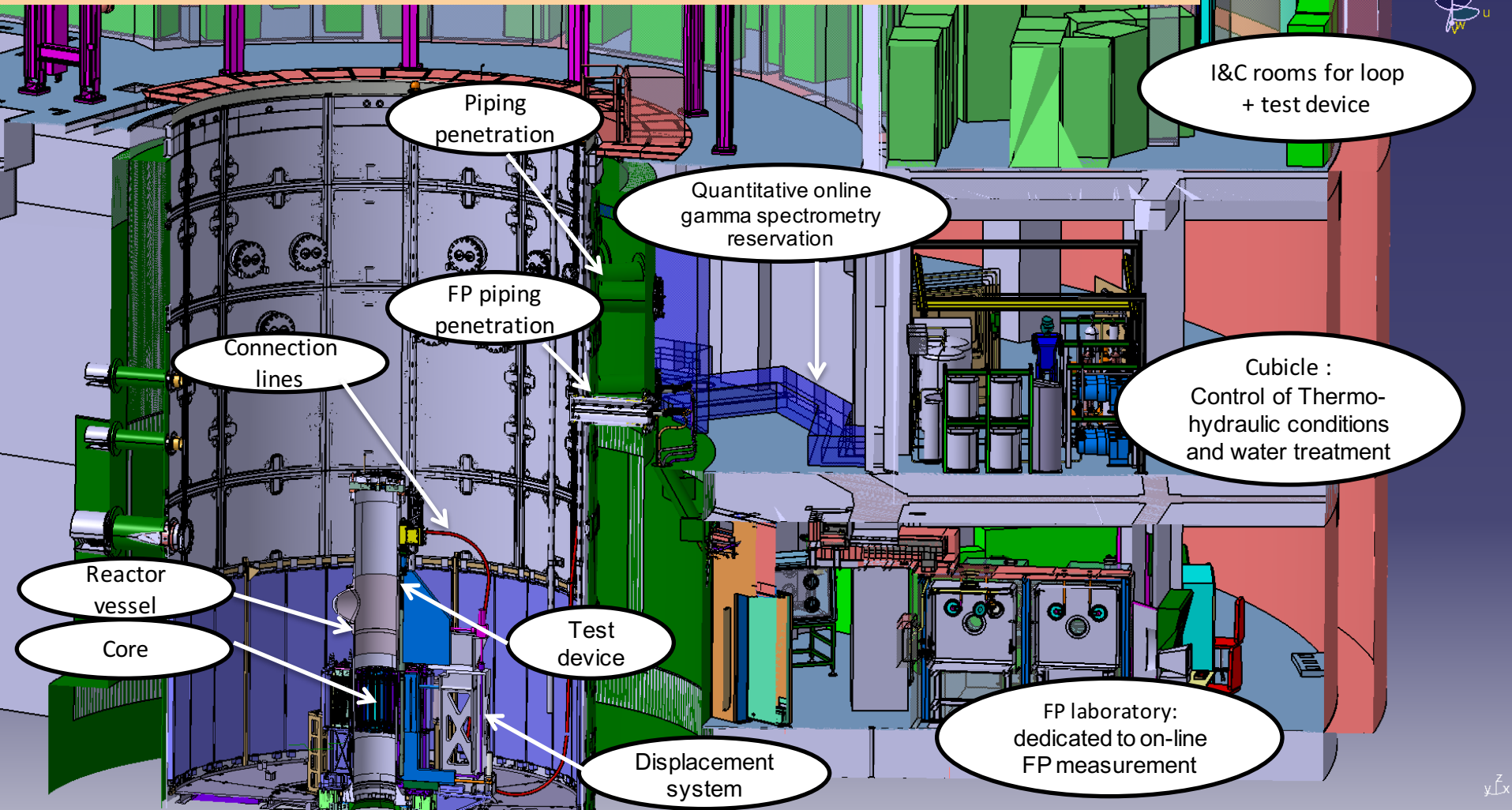


JHR facility & experimental capacity



General architecture of a test device:

in-pile part, lines, out of pile parts (water loop, I&C systems...)

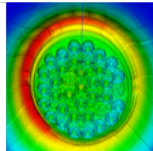


JHR: experimental capacity & performances at 100 MW power level (*)

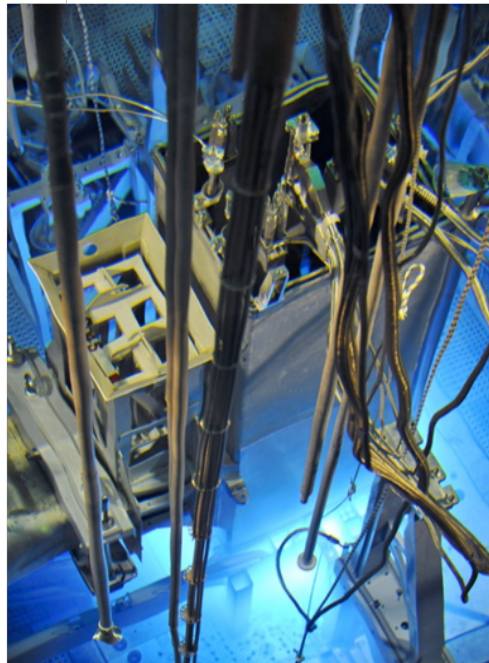
(*) Maximum Power- Second Operating conditions: 70 MW

Thermal Neutrons flux In reflector

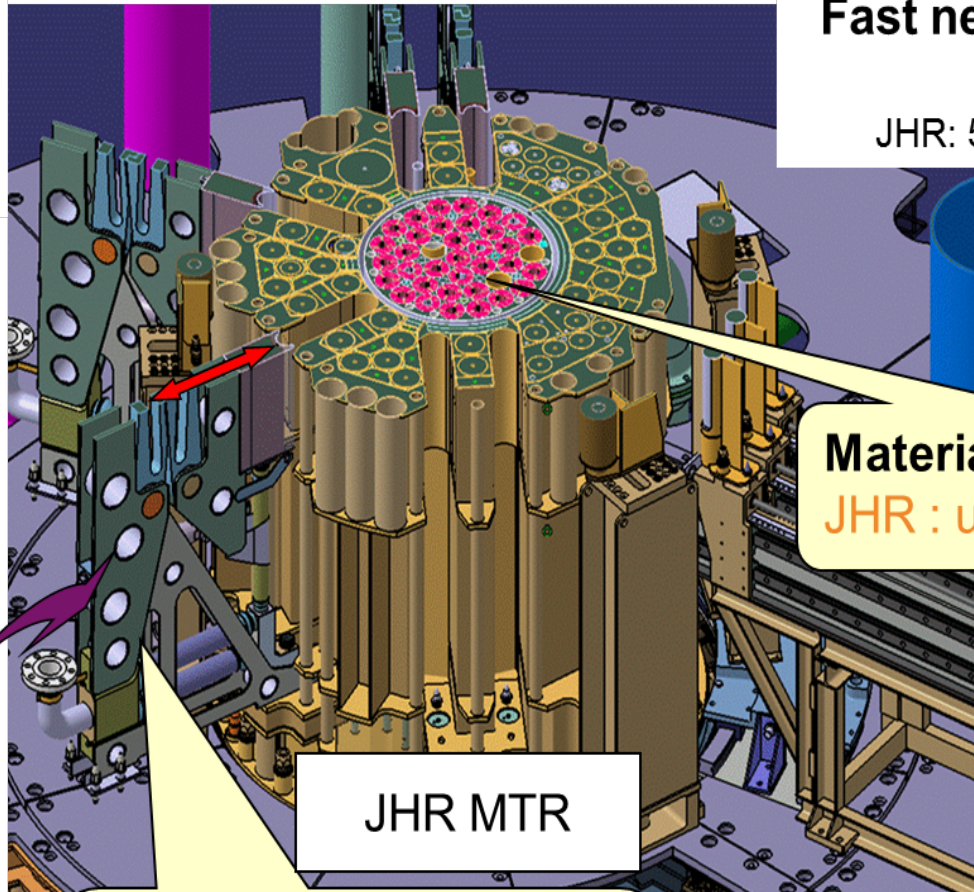
JHR: $5.5 \cdot 10^{14}$ n/cm².s
and 6 displacement systems



Thermal neutron flux



OSIRIS MTR



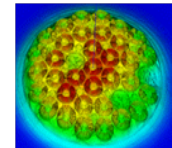
JHR MTR

Displacement systems in
JHR to:

- Adjust the fissile power
- Study transients

Fast neutrons flux in core

JHR: $5.5 \cdot 10^{14}$ n/cm².s



Fast neutron flux

Material ageing
JHR : up to 16 dpa/y

In JHR :

- Highly Instrumented Experiments
- On-line fission Gas analysis
- 20 simultaneous experiments